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# Project X Broader Impacts Spallation & Irradiation Facility

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Community Summer Study 2013 (Snowmass on the Mississippi) 30 July 2013

### Introduction/Summary



- Project X Spallation and Irradiation Facility described
  - Volume 3: Project X Broader Impacts <u>arXiv:1306.5024</u> [physics.acc-ph]
- Content of this volume draws heavily on previous workshops & reports
  - Project X Energy Station (PXES) Workshop, January 29-30, 2013
  - PX Forum on Spallation Sources for Particle Physics, March 19-20, 2012
- The PXES Workshop included participants from the Nuclear Energy and the Fusion Energy community and participants for DOE-NE HQ
- The Project X Energy Station as a Candidate Fusion Materials Facility was submitted to FESAC as part of the SC future facilities exercise
  - Fusion Nuclear Materials Facility deemed "Absolutely Central"
- Briefed both SC-FES and DOE-NE on PX Materials Irradiation Facility
- (Substantial) effort required to evolve the Integrated Target Station (Energy Station + Nuclear Physics) to a pre-Conceptual Design
  - Need to engage DOE-NE, DOE-SC-FES and their research communities

## Project X Energy Station Workshop January 29-30, 2013



PNNL-22263 Prepared for the U.S. Department of Energy under Contract DE-AC05-76RL01830 **Project X Energy Station Workshop** Report Report by the Organizers and Co-Conveners of the Project X Energy Station Workshop David Asner (PNNL) Patrick Hurh (FNAL) Mikey Brady-Raap (PNNL) Bernie Riemer (ORNL) Yoursy Gohar (ANL) Dave Senor (PNNL) Mary Peterson (PNNL) Dave Wootan (PNNL) Eric Pitcher (LANL/ESS) March 2013 Pacific Northwest Proudly Operated by Battelle Since 1965

#### Workshop objectives

- Identify & explore the nuclear and fusion energy relevant R&D that would be possible in an Energy Station associated with the Project X Linac
- Discuss the hypothesis that an Energy Station associated with Project X could accelerate and enhance the ability to test and evaluate early research concepts.
- Identify the synergy and benefit that the Project X Linac could bring to the nuclear & fusion energy communities.

### **Energy Station -> Integrated Target Station**



#### Goal

Develop integrated spallation target station concept to serve DOE-SC-HEP/NP/FES, DOE-NE experimental needs

#### **Rationale**

- CW spallation neutron source could augment limited US irradiation testing capability
- Synergy between Physics experimental needs and materials testing for fusion, fission communities

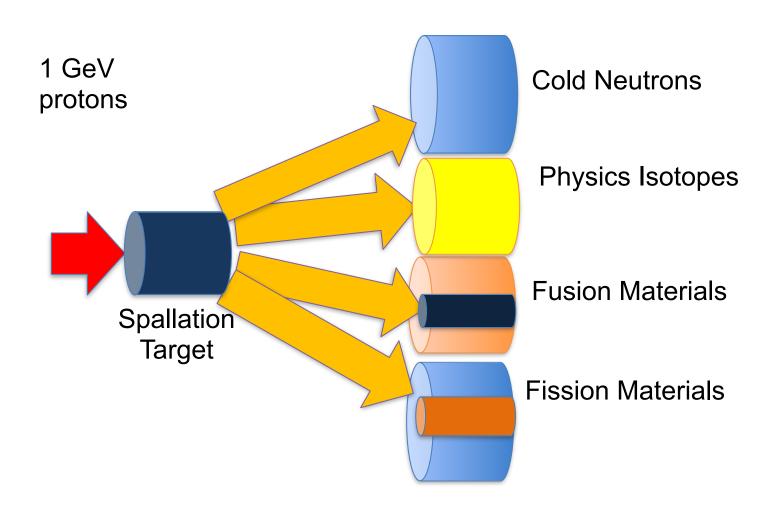
#### Project X - Stage 1

Could provide ~1 MW of beam dedicated to a spallation neutron source for nuclear materials and fuels research (Energy Station) or shared with a physics mission facility with similar neutron source requirements (Integrated Target Station)

## **Energy Station -> Integrated Target Station**



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## Project X Integrated Target Station has the potential to benefit several areas (beyond HEP)



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- ► Highest priority opportunities within the US Nuclear and Fusion energy programs are irradiation of fusion and fast reactor structural materials.
- Must provide a fusion and fast reactor relevant neutron flux at a minimum of 20 dpa per calendar year in a reasonable irradiation volume.
- Enable the in-situ real-time measurements of various separate-effects phenomena in fuels or materials, which would be very valuable to the modeling and simulation technical community. Such capabilities are more feasible in an accelerator-based system than a reactor
- Integral effects testing of fast reactor fuels, including driver fuel, minor actinide burning fuel, and transmutation of spent fuel.
- Support DOE Office of Nuclear Energy plus Office of Science programs
  - Materials Program Fusion Energy Sciences (FES)
  - Isotope Production Program Nuclear Physics (NP)
  - Ultra Cold Neutrons Nuclear Physics (NP)

## **Integrated Target Station Concept**



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#### **Lead Matrix Test Region**

- Solid lead with gas or water cooling
- ~ 2 m diameter, 3 m length
- Low n absorb/ High n scatter
- High n flux/ Fast n spectrum
- Acts as gamma snield

#### **Project X Proton Beam**

1mA @ 1 GeV (1 MW)

#### **Spallation Target**

- Liquid Pb-Bi or Tungsten
- >30 neutrons/proton
- Protons penetrate ~50 cm
- Neutrons ~ fission spectrum
- ~ 10 cm dia, 60 cm length
- Cleanup system for spallation products

#### Reflector •Steel/iron/nickel •High n scatter Flattens n flux distribution **Closed Loop Test Modules** Removable/replaceable/ customizable Independent cooling system n spectrum/material/ temp/pressure to match conditions • ~30 cm dia Moderator tank (if needed) Moderates neutrons for CN **Neutron Beam Tube** Neutrons to N-Nbar, N-EDM

experiments

## Target Station Components – Spallation Target



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- Spallation Target:
  - 6.24e15 p/s proton beam
  - Nominal 10 cm diameter
  - High neutron yield Pb or LBE ~30 neutrons/proton
  - 1 GeV protons penetrate ~50 cm
  - Neutron spectrum similar to fission but with high energy tail
  - Coolant is target material, no stress issues in target
  - Beam window may be life limiting
    - Experience base from ISIS, SINQ, MEGAPIE, SNS, is ~7-22 dpa/yr on front window for SS316, T91, Inconel
    - For 10 cm diameter ESS window, ~8 dpa/yr
  - Need careful oxidation control, on-line cleanup
  - Spallation products like fission products
  - >400 KW energy deposited
  - Potential for in-beam materials testing

MEGAPIE (0.8 MW) LBE target has been demonstrated

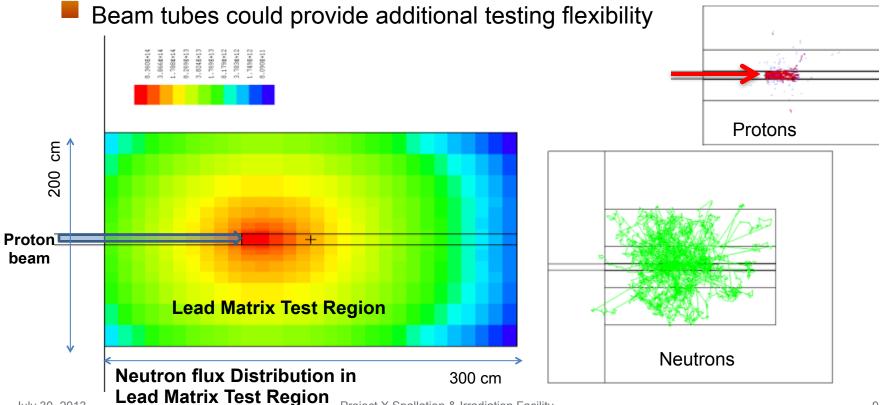


### **Target Station Components – Test Matrix**



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- Test Matrix
  - Solid lead or other (e.g. Zr-base alloy) high scatter, low absorption
  - Maximizes neutron flux, provides space for array of test modules
  - Simple thermal analysis indicates heating may allow solid lead matrix



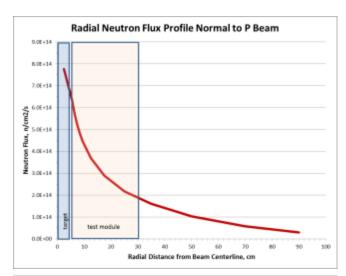
## High Flux Volumes Available in Test Matrix Region

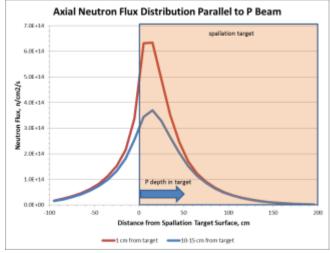


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Neutron Flux Range (n/cm2/s)	Axial Extent (cm)	Outer Extent (cm)	Volume (liters)
>5e14	<i>30</i>	8	~2.8
>3e14	<i>50</i>	15	~23
>1e14	110	60	~600
>5e13	160	<i>80</i>	~2000
>1e13	250	100	~9000

- ▶ 1 GeV protons penetrate ~ 50 cm in lead or LBE target, generate ~30 neutrons/proton
- Neutron flux falls off radially but lead matrix helps
- Axial profile peaks ~20 cm below target surface, provides ~100 cm >1e14 n/cm²/s

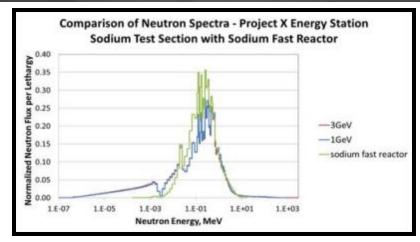




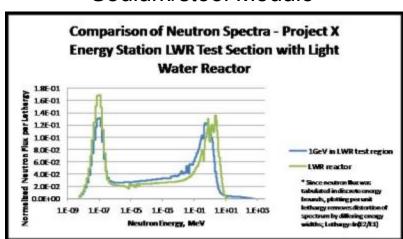
### **Spectrum Tailoring Can Simulate A Different Reactor in Each Module**



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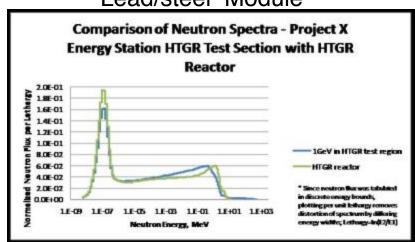


#### Sodium/steel Module



Comparison of Neutron Spectra - Project X **Energy Station LFR Test Section with LFR Reactor** 4.0E-01 3.5E-01 3.0E-01 2.5E-01 2.0E-01 1GeV in LFR test region 1.5E-01 ead Fast Reactor 1.0E-01 5.0E-02 0.0E+00 1.E+01 energy widths, Neutron Energy, MeV Letharpy Int 7/11)

#### Lead/steel Module



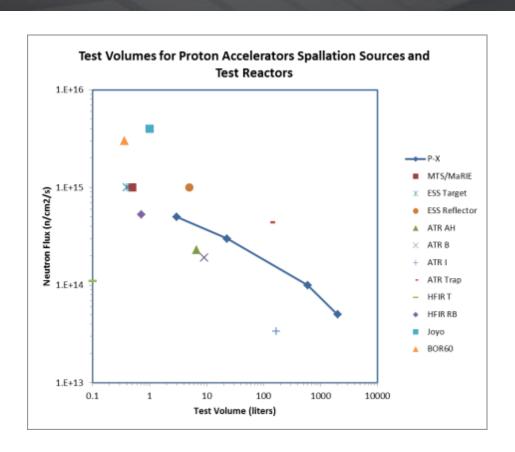
Water/Zr Light Water Module

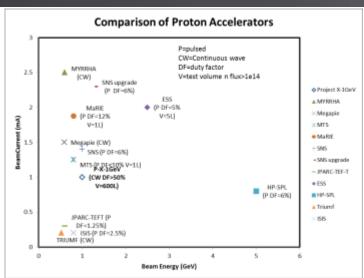
Graphite/He Module

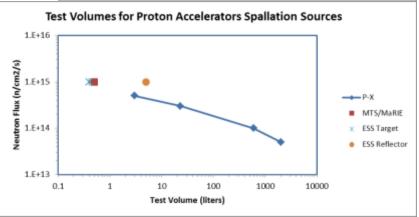
## **How Does Target Station Compare?**



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- Irradiation volumes at high flux comparable to reactors
- Accelerator parameters are in range of other proposed systems

## Actions identified to evolve concept



- Develop conceptual target designs that serve both particle physics and nuclear energy missions – Integrated Target Station (ITS)
- Develop an ITS testing program plan that capitalizes on the unique characteristics of a high-intensity accelerator and spallation source
- Define/refine the technical requirements to support the proposed testing program plan
- Compile relevant design parameters to support the high-priority mission needs and provide them to the beam and target designers
- Investigate the beam on/off issues for both short and long time scales. to determine which transients have the potential to be problematic due to thermal and radiation damage effects
- Further consideration must be given to desired damage rate/sample volume specifications to provide a meaningful irradiation capability
- Neutronics modeling of the notional Project X ITS concept needs to be refined to evaluate beam options (e.g., dual or rastered beam) to optimize flux and flux gradients in maximum usable test volumes.

## **Backup Slides**



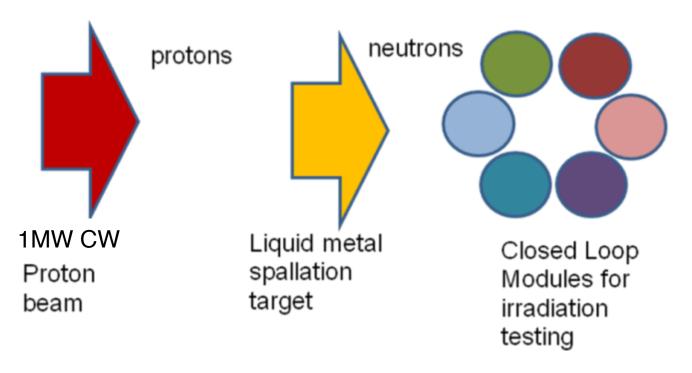
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## **PNNL Target Station Concept**



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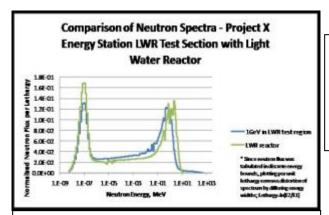
A new approach utilizing the flexibility of an <u>accelerator neutron</u> <u>source</u> with <u>spectral tailoring</u> coupled with a careful design of a <u>set of</u> <u>independent test loops</u> can provide a flexible neutron test station for DOE applications



### **Project X Energy Station Concept**



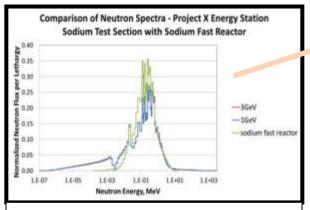
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### **MSR**

Thermal Spectrum Test Module: LWR, HTGR,

#### **Project X Proton Beam** 1mA @ 1 GeV (1 MW)



Fast Spectrum Test Module: SFR, LFR, GFR

#### **Closed Loop Test Modules**

- Removable/replaceable/customizable
- Independent cooling system
- n spectrum/material/temp/pressure to match reactor conditions
- ~30 cm dia

#### **Lead Matrix Test Region**

- Solid lead with gas or water cooling
- ~ 2 m diameter, 3 m length
- Low n absorb/ High n scatter
- High n flux/ Fast n spectrum
- Acts as gamma shield

#### **Spallation Target**

- Liquid Pb-Bi
- >30 neutrons/proton
- 1 GeV protons penetrate ~50 cm in lead
- Neutrons Similar to fission spectrum
- Samples can be irradiated in proton beam
- Adding W or U can increase n flux density
- Small volume ~ 10 cm dia, 60 cm length
- Cleanup system for spallation products

#### Reflector

- Steel/iron/nickel
- High n scatter

## Target Station is Unique Combination of Existing Technologies

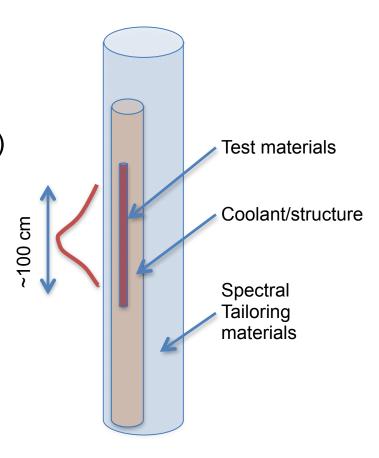


- Proton beam CW 1 GeV 1 mA 1 MW
- Spallation Target:
  - Liquid lead or lead-bismuth release ~30 neutrons/proton
  - Neutron spectrum similar to fission spectrum but with high energy tail
  - Technology has been demonstrate at MEGAPIE
- Test Matrix
  - Solid lead or other (zircalloy) high scatter, low absorption
  - Maximizes neutron flux, provides space for array of test modules
  - Simple solid block with cooling, holes for test modules
- Closed Loop Test Modules
  - Independently tailored irradiation environments (LWR, HTGR,SFR,LFR)
  - Independent heating/cooling system for each to control temperatures
  - Concept utilized in FFTF (sodium), BOR-60 (sodium, lead), ATR (press. Water)
- Reflector to minimize leakage neutrons

## Target Station Components – Closed Loop Test Modules



- Number of modules can be varied
- Each module can have unique independent coolant and materials and operate at independent temperatures (sodium, lead, molten salt, water, helium)
- Neutron spectrum can be tailored from fast to thermal to match reactor conditions (the gamma to neutron ratio can also be tailored)
- Miniaturized test specimens can maximize testing in high flux region
- Modules are Removable, Replaceable, shipped offsite for post irradiation examination (PIE)

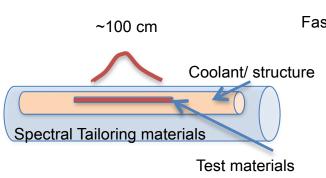


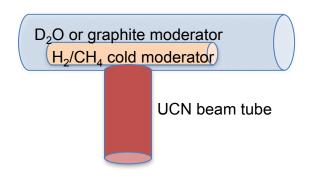
### **Closed Loop Modules**



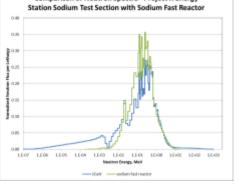
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- Each module can have unique independent coolant and materials
- Removable/ replaceable/ customizable
- Independent cooling system
- n spectrum/ material/temp/ at desired conditions
- ~30 cm dia, 100 cm long test region

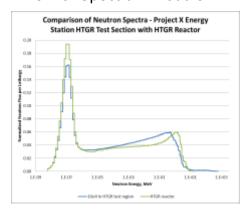




#### Fast Spectrum Materials Testing Module Comparison of Neutron Spectra - Project X Energy Station Sodium Test Section with Sodium Fast Reactor



#### Thermal Spectrum Module







R&D Scope	Environment	Testing Needs
Technology gaps requiring materials qualification • Plasma facing components • Low activation	<ul> <li>14 MeV neutrons</li> <li>Materials     surrounding fusion     ignition region</li> <li>Tritium producing     lithium blanket</li> </ul>	<ul> <li>Structural material properties as a function of dpa and temperature</li> <li>20-30 dpa/yr</li> <li>Cumulative 150-200</li> </ul>

## Target Station Can Help Fusion Energy Science Technologies



- Dedicated fusion loop for materials testing with high energy neutron spectrum test environment at relevant temperatures
- Room for separate lead, helium, water loops that can be used to simultaneously test materials interactions
- The dpa accumulation and high energy neutron spectrum component can simulate fusion environments better than reactors
- Testing in the proton beam can provide high dpa and high neutron energies
- H and He generation rates for corresponding damage accumulation could allow testing of fusion materials
- Candidate fusion blanket materials can be irradiated in a prototypic coolant, temperature, and high neutron flux
- Temperature is a critical parameter in materials irradiation and precise temperature control will be a key aspect of the Energy Station Test Module design





R&D Scope	Environment	Testing Needs
<ul> <li>Source of Ultra cold neutrons for n-EDM, NNbar</li> <li>Source of isotopes for ISOL atomic EDM</li> </ul>	<ul> <li>Cold Neutrons</li> <li>Very Cold Neutrons</li> <li>Ultra Cold Neutrons</li> <li>Well moderated neutron spectrum</li> </ul>	<ul> <li>Stable, well characterized test environments</li> <li>UCN n velocities &lt;4mK</li> <li>UCN density &gt;3x10<sup>4</sup> UCN/cm<sup>3</sup></li> <li>Beam tubes</li> </ul>

## Target Station Can Help Nuclear Physics Technologies



- Separate closed loop with heavy water, Be, metal hydride, moderator region,
- Cryogenic cooled He, H2,HE-2, CH4 volume for producing Ultra cold neutrons,
- Reflected Cold Neutron beam transport to n-EDM, NNbar experiments
- Possibility for irradiating Thorium spallation target capsules in proton beam region to produce ISOL isotopes





R&D Scope	Environment	Testing Needs
Radioisotope Production methodology	<ul> <li>Neutron spectrum tailored for specific isotopes</li> <li>Easy access and retrievability</li> </ul>	<ul> <li>Low activation target and structural materials</li> <li>Target/capsule compatibility</li> </ul>

## Target Station Can Help Isotope Production Technologies



- Dedicated isotope production loop with capability to vary neutron spectrum test environment and temperatures to optimize for isotopes of interest
- Room for separate loops that can be used to simultaneously produce and test a variety of isotopes
- Testing in the proton beam can provide accelerator produced isotopes
- Higher neutron energies than reactors can enhance production of isotopes only produced by fast neutrons
- Temperature is a critical parameter in some isotope target irradiations and precise temperature control will be a key aspect
- A rabbit system can be integrated into the test module for rapid insertion and retrieval of short lived radioisotopes
- Reflector region can utilize "waste" neutrons for beneficial isotope production such as <sup>60</sup>Co
- Spallation reactions produce broad range of reaction products and the target cleanup system could be designed to separate particular isotopes of interest

## DOE Office of Nuclear Energy Fuel Cycle R&D Advanced Reactor Technologies



R&D Scope	Environment	Testing Needs
<ul> <li>Advanced Reactor Technologies</li> <li>Gen-IV, aSMR</li> <li>Basic physics</li> <li>Material research and testing</li> <li>Modeling and simulation of reactor systems and components</li> <li>Probabilistic risk analysis of innovative safety designs and features</li> <li>Development activities to establish concept feasibility for future deployment</li> <li>Fuel Cycle R&amp;D</li> <li>Material testing</li> <li>Structural materials</li> <li>Advanced nuclear fuels</li> <li>Reactor systems</li> <li>Instrumentation and controls</li> <li>Power conversion systems</li> <li>Process heat transport systems</li> <li>Dry heat rejection</li> <li>Separations processes</li> </ul>	<ul> <li>Sodium cooled fast reactor (SFR)</li> <li>Sodium coolant Steel cladding</li> <li>Temperature range ~550 C</li> <li>Lead cooled fast reactor (LFR)</li> <li>Fast spectrum</li> <li>Pb or LBE coolant Steel cladding</li> <li>Temperature range 500-800 C</li> <li>Gas cooled fast reactors (GFR)</li> <li>Fast spectrum</li> <li>Temperature range ~600-850 C</li> <li>Helium coolant Steel cladding</li> <li>Very High Temperature Reactor (VHTR)</li> <li>Thermal spectrum Graphite moderated</li> <li>TRISO fuel Helium cooled</li> <li>Temperature range ~1000 C</li> <li>Supercritical water cooled reactor (SCWR)</li> <li>Thermal spectrum</li> <li>Light water coolant Steel cladding</li> <li>Temperature range ~550 C</li> </ul>	<ul> <li>Structural material properties as a function of dpa and temperature</li> <li>Material compatibility at operating conditions</li> <li>Integral tests of fuel, structural materials</li> <li>Feature tests of components</li> <li>Fuel performance with minor actinides</li> <li>Cumulative ~200 dpa</li> <li>Long life fuel concepts ~300-500 dpa</li> </ul>

Waste forms

system elements

Modeling and simulation

Small scale tests to validate

Sodium fluoride salt coolant

Molten Salt Reactor (MSR)

Thermal spectrum

## Target Station Can Help Fuel Cycle R&D and Advanced Reactor Technologies



- All of the advanced concepts can be tested in appropriately designed test loops
- Neutron flux tailoring can create fast or thermal spectrum test environments that match very different reactor environments at relevant temperatures
- Room for separate sodium, lead, LBE, helium, molten salt, water loops that can be used to simultaneously test materials interactions
- Candidate materials such as fuel and cladding can be irradiated in a prototypic coolant, temperature, and neutron spectrum
- Temperature is a critical parameter in materials irradiation and precise temperature control will be a key aspect of the Energy Station Test Module design
- Higher H and He generation rates for corresponding damage accumulation could allow accelerated testing of materials
- Potential test volumes in Target Station could be 100's of liters

### **Target Station Capabilities**



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- Flexible design allows support to multiple missions for DOE-SC-HEP, SC-NP, SC-FES, DOE-NE
- Benefits of test reactor volumes and neutron fluxes without reactor issues – licensing, fuel supply, safety, waste
- Robust technology allows it to be designed and constructed with today's technology in order to fill gaps in tomorrow's technology
- Continuous wave, high availability, high beam current provides potential for irradiation tests to high fluence

- Energy distribution of spallation neutrons similar to fast reactor fission spectrum but with high energy tail up to proton energy
- Ability to tailor neutron spectrum from fast to thermal as well as the gamma to neutron flux ratio
- H and He generation in materials higher than in reactor allowing accelerated aging testing
- Potential for beneficial isotope production and/or neutron beams simultaneous with irradiation testing

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